



Topic of FATIMA project

"(Environmentally-assisted) fatigue* management of austenitic stainless steels in NPP primary circuit for long-term operation"

^{*}Progressive localized permanent structural change occurring in a material subjected to conditions that produce fluctuating stresses and strains at some point and that may culminate in cracks or complete fracture after a sufficient number of fluctuations (ASTM E1823)

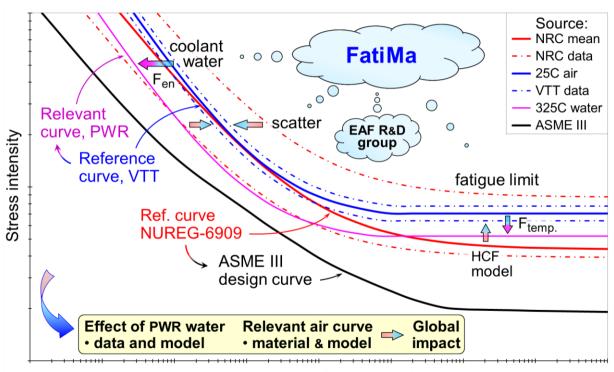


Project motivation

- Accumulation of (calculated) fatigue usage as a potentially limiting factor for power plant use
 - Realism needed in fatigue management
 - → reduce cumulative usage and maintain/improve safety
 - Avoid bias in risk-informed in-service inspection and plant life management
 - → focus in critical points and optimize radiation exposure
- Introduction of scientific-based methods into codes and standards
 - Recognition and impact through international collaboration
 - Understand and improve state-of-the-art, not only follow



Project structure



Allowable number of transients / cycles



Achievements in 2020



WP1 – State-of-the-art & internat. collaboration

- Focus area of the first project year and baseline for other WPs in 2021–22
- State of the art of EAF rules reviewed in ASME PVP2020 conference paper
 - Sequel submitted to ASME PVP2021
 - Task has been completed
- EPRI EAF Collaboration Group
 - 'Team Finland' membership accepted
 - Two webinars in 2020, to continue in 2021
- Other participation e.g. ASME Code Week meetings etc.

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CODES, STANDARDS, RULES AND ASSUMPTIONS ON ENVIRONMENT ASSISTED FATIGUE FOR FATIGUE MANAGEMENT OF PRIMARY PIPING

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ABSTRACT

All international codes used for design, operation and inspection of NPP primary circuit pressure boundaries are rooted to the ASME Boiler and Pressure Vessel Code, Section III, Nuclear Vessels, 1963. Article 4, N-415 "Analysis for cyclic operation" instructed calculation of stress intensities for futigue transients and provided two design curves for basic material types. Different codes such as ASME, RCC-M, KTA, PNAE and JSME have much in common, but partial deviations exist.

In 2007 the US NRC Regulatory Guide 1.207 endorsed a methodology for accounting the environmental effects. It was mainly based on extensive work in Japan and the Argonne National Laboratory. The final report of ANL, NURGO(CR-6909) became a major reference and subject of criticism. However, the first approach for environment assisted Jatque (EAF) written in code language was published in Japan and a regulatory requirement for consideration of EAF both for operating reactors and new designs appeared first in Finland.

This paper discusses challenges in management of fatigue and the evolving state-of-the-art in different codes, standards, rules and assumptions. The roots and current status of fatigue curves and design criteria applied in Finnish NPP's are evolained.

Keywords: codes and standards, fatigue, stainless steel, EAF

NOMENCLATURE

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 $F_{en,T}$ partial factor for isolated temperature effect $F_{en,water}$ partial factor for isolated water effect

HCF high cycle fatigue
JSME The Japan Society for Mechanical Engineers
KTA Kerntechnisher Ausschuss

LCF low cycle fatigue LTO long term operation (of reactor)

LWR light water reactor
N, N_(1"n") number of cycles, f=at failure, "n"=environment
N_{25,T} number of cycles at specified temperature T

N25,T number of cycles at specified to
NPP nuclear power plant
PWR pressurized water reactor
RA reduction of area

RT room temperature S_a [allowable] stress intensity amplitude, ε_a ·E

 S_a (allowable) stress intensity amplitude, $E_a \cdot E$ S_{alt} alternating part of stress intensity (from stress analysis) S_a stress intensity amplitude at endurance limit

 Γ temperature Γ plastic strain range

 ε_a total strain amplitude

1. INTRODUCTION

Environmentally assisted fatigue (EAF) is an issue that cannot be ignored in design and ageing management of NPP primary piping components, which are in contact with the reactor coolant.



WP2 – Quantitative environmental effects WP3 – Fatigue design curves

- Experimental work packages (in reactor coolant and in air)
- Scaling up of activities in 2021, preparatory work started in 2020
- Nuclear grade stainless steel → maximum relevance for Finnish NPPs
- Revitalization of experimental research in simulated PWR water



Thank you for listening!

